Synthesis of Calculational Methods for Design and Analysis of Radiation Shields for Nuclear Rocket Systems

The problem:
To devise a computer program for the complete initial and final design of nuclear rocket radiation shields.

The solution:
Eight computer programs making up a nine volume synthesis containing two design methods. The “early design” method is appropriate for parametric and preliminary reactor design studies, while the “final design” method accomplishes the verification of a final nuclear rocket reactor design.

How it’s done:
Both design methods begin with the initial program, POINT, which contains the basic microscopic cross section library. In addition to POINT, the early design method contains three programs. Its important features are: flexibility in geometry and usage; convenience and usefulness of output; and nominal running time. The final design method, made up of five programs plus POINT, is highly accurate and has detailed output such as energy and angular distributions.

The eight programs used are:
(1) POINT (Preparation of Input for Transport) Program. This program calculates macroscopic cross sections readily usable in the transport programs, TAPAT and ODD-K. This calculation is achieved by employing a built-in library of region independent neutron and gamma ray microscopic cross sections. The program also calculates response functions as punched card output for use in both the TAPAT and NAGS programs to calculate, for example, neutron and gamma ray energy deposition, fixed neutron sources, or prompt and secondary gamma ray sources.

(2) TAPAT (The Analysis Program and Transport) Program System. This program system provides a complete, neutron and photon, one-dimensional radiation analysis of a reactor system in a single computer run.

(3) KAP-V (Point Kernel Attenuation) Program. This program employs the point kernel method to calculate radiation levels at detector points located within or outside a complex radiation source geometry describable by a combination of quadratic surfaces. This program can be used, for example, to calculate gamma ray and/or fast neutron flux, dose, or heating rate. Three optional fast neutron attenuation functions are included: (1) a modified Albert—Welton function; (2) a bivariant polynomial expression for computing neutron spectra using infinite media moments data; and (3) a monovariant polynomial expression for computing neutron spectra using infinite media moments data. The program handles either cylindrical, spherical, disk, line or point sources.

(4) TIC-TOC-TOE (Temperature in the Coolant Tank and Other Calculations and for the Thermal Neutron Originating Energy) Program. This program performs rapid calculations of heating rate distributions in on-axis liquid hydrogen propellant tanks. Basic heating rate data are interpolated from curve-fits of M. O. Burrell’s Monte Carlo data which are built into the program.

(5) ODD-K (Two-Dimensional Transport) Program. This program is the Westinghouse Astronuclear Laboratory modification of the Los Alamos Scientific Laboratory discrete ordinate transport program, DDK. The ODD-K program numerically solves the Boltzmann transport equation using a multigroup, discrete...
direction representation of the particle flux in a two-
dimension (R–Z, R–θ, or X–Y) geometry description
of a reactor. ODD-K performs photon, as well as
neutron, transport analysis and, in conjunction with
the NAGS and DAFT codes, it provides a method for
obtaining neutron and photon spatial and energy dis-
butions throughout the reactor system, and spatial,
energy, and angular distributions of leakage fluxes at
the surface of the reactor system.

(6) NAGS (Calculations of Neutron and Gamma
Ray Sources and Energy Deposition in Two-Dimen-
sional Geometries) Program. This program is a series
of routines which process multigroup neutron and
photon energy fluxes for two-dimensional (R, Z or R,
θ) geometry models. The NAGS program provides:
(1) neutron and photon energy sources and distribu-
tions for use in point kernel, Monte Carlo, and photon
transport analyses, (2) neutron and gamma ray dose
rates, and (3) energy deposition data for use in sub-
sequent thermal analyses.

(7) DAFT (Preparation of ODD-K Angular Flux
Tapes) Program. This program is the data processing
routine which prepares angular, spatial, and energy
distribution data for input to the FASTER Monte
Carlo program from the surface angular leakage flux
data of the two-dimensional transport program
ODD-K.

(8) Faster Monte Carlo (A Fortran Analytic Solu-
tion of the Transport Equation by Random Samp-
ing) Program. This program calculates energy-dependent
neutron or photon fluxes at points, surfaces, and
regions of complex geometries. FASTER contains all
the data processing routines required for a wide variety
of nuclear vehicle applications. FASTER deals with
the entire spectrum of particle energies simultane-
ously.

Notes:
1. This program is written in FORTRAN IV and
MAP for use on IBM 7094 computer.
2. Although this set of programs has been designed
primarily for use by personnel involved in the
design and analysis of nuclear rocket radiation
shields, some of the individual programs can be
used by scientists involved in basic nuclear and
nuclear reactor research.
3. Inquiries concerning this program may be
made to:

   COSMIC
   Computer Center
   University of Georgia
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   Reference: B69-10158

Patent status:
No patent action is contemplated by NASA.

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Westinghouse Astronuclear Laboratory
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